

REDUCTION OF

RADIOLOGICAL

CONSEQUENCES

ACCIDENT

Overview of the EU H2020 R2CA project:

Reduction of Radiological Consequences of Accidents

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REDUCTION OF RADIOLOGICAL CONSEQUENCES OF DESIGN BASIS & DESIGN EXTENSION ACCIDENTS

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Origin & Context



- Issued from 4th periodic safety review of French 900 MWe PWR but also worldwide due to more challenging safety objectives targeted for NPPs
 - ❖ Increased NPPs safety required in terms of RC reduction in addition to safety criteria in safety accident analyses
 - ⇒ For DBA : tend to reduce the RC down to levels where no more population protection measures
 - ⇒ Considering scenarios more severe than DBA (incl. DEC-A situations)
- After FKS large R&D efforts on the evaluation of Severe Accident progression and consequences lead to increase the predictability of SA progression, evaluate AMPs and improve their consequence mitigation
 - Less efforts paid on DBA and DEC-A situations leading to reduce the differences in risks associated to DBA and BDBA
- DBA largely studied and modeled for establishing safety criteria to fulfill safety objectives, but evaluations
 usually done with very conservative deterministic assumptions/decoupled approaches and fewer efforts paid to
 best estimate evaluation of RC
 - For LOCA DBA objectives related to clad integrity and core coolability and RC evaluated with specific decoupled assumptions (i.e. in France 33% of the fuel rods assumed to fail)
 - ⇒ Optimisation of EP&R actions + quantification of gains (in terms of RC) of additional measures/devices impossible

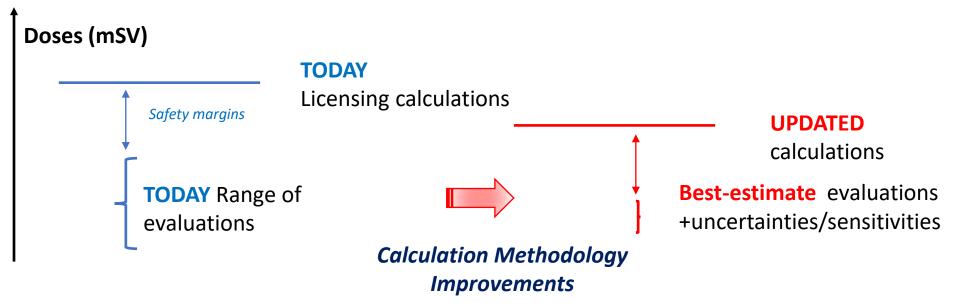




Motivation



 Refine the assessments of radiological consequences of explicit DBA and DEC-A accidental scenarios in Gen II, Gen III and Gen III+ NPPs focusing on LOCA and SGTR scenarios



- Increase the level of NPP safety by more realistic evaluations of DBA
 - **FKS studies evidenced the decrease in** differences between evaluated risks for DBA and BDBA conditions and the use of very conservative assumptions/tools for RC evaluations of DBA
- Strengthen assessments of NPP safety levels considering situations more severe than those integrated in plant designs (DEC-A domain ← wo "significant fuel melting")





Objectives & Methodology



Specific objectives of R2CA for a best estimation of LOCA & SGTR radiological consequences

- Upgrade simulation tools (integral, detailed) & improve calculation schemes (coupling...)
- Harmonize calculation methodologies: derivation of some principles for EP&R action optimization
- Optimize accident management procedures (development of new/improved algorithms...)
- Evaluate innovative systems (ATF, safety devices, algorithms...)
- Develop innovative tools based on AI for earlier diagnosis and accident anticipation (SGTR)

Methodology

- Review of existing base (methodologies, simulation schemes, calculation tools, experimental data)
- Identification of reactor cases covering all aspects (conditions, accidents, reactors) & simulation scheme needs
- Upgrade of simulation schemes through re-assessments of experimental data and model improvements
- Identification of gains through comparison of initial/final reactor case calculations (+ uncertainty)
- Evaluation of innovative systems (ATF, AMP algorithms...) with improved calculation schemes



Focus on: PWRs/EPR/VVERs/BWR, LOCA/SGTR, DBA/DEC-A, Radiological Consequences, AMPs



Methodology



Approach for LOCA reactor calculations

Agree on Accidental Scenarios for each reactor type



Perform simulations according to the **conservative approach** (regulatory practices in each country, decoupling factors)

- 1- Improve failed rod number prediction: statistical approach, core modelling refinement
- 2- Improve fuel rod failure models: clad burst criteria update
- 3- Improve FP-release/transport models: consideration of HBS, fuel oxid./frag...

Improvements

Reduction of conservatism (decoupling factors)

Perform simulations according to a **more** realistic (≈ Best-Estimate) approach

same scenarios with improved calc. schemes



Quantify the gains through RC evaluation of the new method compared to conservative approach



Evaluate innovative systems (ATF...)



Uncertainty evaluation



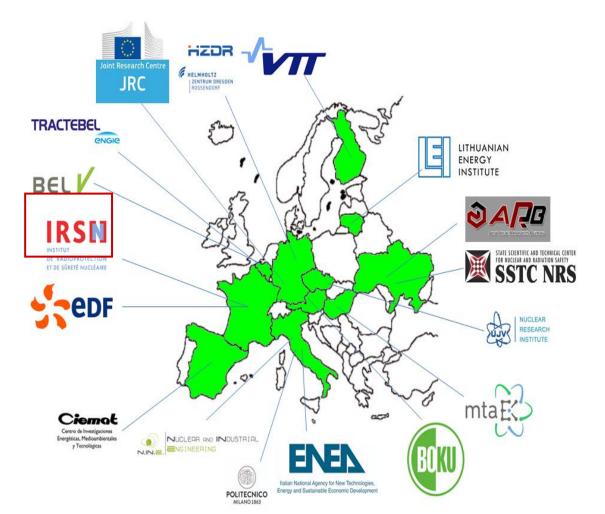


IRSN

Structure

Ressources 17 organisations

Time frame 01.09.2019 – 31.08.2023









Structure



PROJECT MANAGEMENT IRSN DISSEMINATION EDUCATION & TRAINING ENEA METHODOLOGY Simulation tool improvements Review of data tools **Best Estimate reactor** LOSS OF COOLANT ACCIDENT & methodologies calculations Fission product transport and behaviour in containment Accurate evaluation of clad failures in a reactor core Fission product releases from oxidized & high-burn-up fuel Accidental scenario Uncertainty identification quantification STEAM GENERATOR TUBE RUPTURE Iodine transport & release from failed steam generator Initial reactor Methodology lodine release spike from defective rods calculations harmonisation Defective Fuel rod behaviour, clad hydriding EK**INNOVATION** Accident management procedure improvements **Expert system elaboration Promising ATF evaluation** EDE **R&D** activities

Reactor cases simualtions



Structure – WP3 LOCA



Improve calculation tools for more realistic evaluations of LOCA DBA & DEC-A csqces

- Identifying main relevant phenomena
- Developping models, coupling tools to reduce the conservatisms
- Verifying/validating models & tools within the range of DBA/DEC-A conditions on the selected exp. database



AMBITION

Better evaluation of environt source term for more appropriate decisions in EP&R.



Upgrading of models for FP transport in primary circuit (incl. FP behaviour (iod.) in containment)

T3.2 « Clad burst failure»

AMBITION

More accurate evaluation of the number of fuel rod burst failures



Elaboration of new <u>clad burst</u> <u>criteria</u> (revisitation of the exp. database) and core modelling approach

T3.3 « Fuel rod T/M & FP releases»

AMBITION

Refined evaluations of FP release amount & kinetics from fuel



Improvt of FP release models from fuel (incl. chemistry) & <u>coupling of FP release-fuel T/M tools (incl. fuel oxidation, high BU fuel formation)</u>





Structure – WP4 SGTR



Improve calculation tools for more realistic evaluations of SGTR DBA & DEC-A csqces

- Identifying main relevant phenomena
- Developping models, coupling tools to reduce the conservatisms
- Verifying/validating models & tools within the range of DBA/DEC-A conditions on the selected exp. database



T4.1 «« FP releases from primary circuit»»

AMBITION

Better evaluation of environt source term for more appropriate decisions in EP&R.



Upgrading of models for FP releases (esp. iodine) from primary circuit to environment (incl. T/H flashing, carry-over)

T4.2 « FP releases from defective rods» AMBITION

More accurate evaluation of primary circuit activity evolution in NO & transients



Elaboration of models for FP releases from defective fuel rods (incl. spiking release phenomena, fuel leaching)

T4.3 « Clad secondary hydriding & failure» AMBITION

Evaluation of the risks of defective fuel rod failure



Developt of integrated models for clad secondary hydriding in defective fuel rods & clad failure criterion



R2CA Summer School, July 4-6th, Bologna



Structure – WP5 INNOV



Develop technological innovations for reduction of RC of DBA & DEC-A

- Accelerating integration of technological breakthroughs (providing global and long-term vision)
- Identifying and tackling technological and scientific challenges



T5.1 « Pro and Cons of innovative devices and management approaches » **AMBITION**

Best measurement at the right spot for better decisions.



Improvement of AMP: New instrumentation, optimized procedure, neural network



In real time, collect, analyse, harvest values of all operating parameters plants to optimise their safety



Elaborate an expert system based on AI for identification of rod defects from RCS activity variation

T5.3 « E-ATF Anhanced accident tolerant fuel »

AMBITION

Give all nuclear power plants benefit of a fuel that can even better withstand accidents.



OBJECTIVES

Evaluate promising ATF (sensitivity analysis on relevant parameters): focus on Cr-coated Zr clad and Cr-doped & highdensity fuels





Summary



- R2CA project devoted to the elaboration of generic & more realistic methodologies for radiological consequence evaluations of LOCA and SGTR transients in DBA & DEC-A conditions focussed on:
 - Upgrading, coupling & validation of reference tools (detailed and/or integral) from fuel behaviour to environmental FP releases
 - Harmonization of the calculation methodologies applicable to all existing European NPP designs (PWR, BWR, VVER)
- Innovative actions also investigated for their potential applications to reduce the radiological consequences of LOCA/SGTR within those conditions
 - Evaluation of near-concept ATFs for LOCA with updated calc. schemes (sensitivity analyses on material base properties)
 - Investigation/evaluation of innovative devices/systems (algorithms), optimisation of accidental management procedures (additional water injection, dedicated instrumentation...)
 - Use of "Artificial Intelligence" for anticipation of SGTR accidental sequences (turn the prognosis tools into diagnosis ones for early detection of defective fuel rods)



Thank you!



